

ICAMCYL

Centro internacional de materiales avanzados y materias primas
International center for advanced materials and raw materials



Advances in multi-scale modelling approaches of liquid metals and alloys for advanced nuclear applications

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Reunión Española de
"Modelado, caracterización y desarrollo de aleaciones metálicas para aplicaciones nucleares"
Madrid, 27-28 de Febrero de 2019

1. ABOUT ICAMCyL

2. ICAMCyL & MATERIALS DISCOVERY

3. PREVIOUS PROJECTS

4. CURRENT PROJECTS

ICAMCyL, A REFERENCE COMPETENCE CENTER

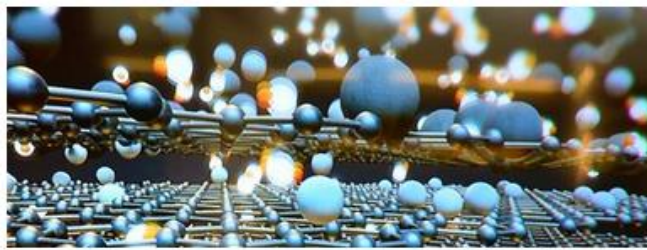
Non-profit private **research foundation** and **competence center**, founded by some of the main industries from the CyL region in the sectors of advanced materials, engineering, mining and processing and automotive, with the support of the regional government of Castilla y León and the County Government of León.

Key player in the European strategy for the efficient management of industrial resources, energy efficiency, eco-innovation and substitution of critical raw materials with the aim of **promoting the development** of advanced materials for the regional network of industries and the **valorization** of the Castilla y León richness in raw materials, in line with its **Smart and Intelligent Specialization Strategy (RIS3)**.



SUSTAINABLE MINING

The development of a sustainable and low environmental impact 21st century mining through the development and integration of novel methods, techniques and processes that allow the utilisation and valorisation of raw materials and subproducts, always in agreement with the principles of sustainability and circular economy.

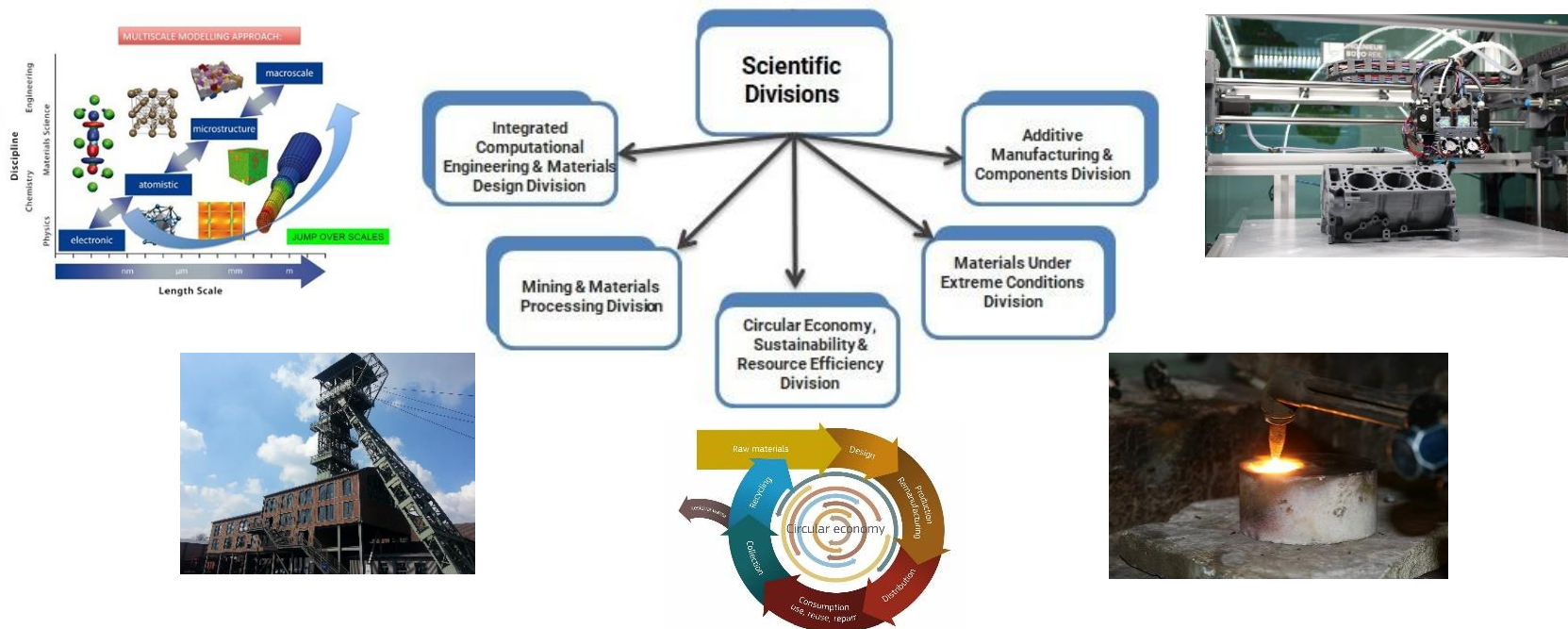


ADVANCED MATERIALS

The development, preparation and fabrication of the last generation novel advanced materials with high added-value, and their application in products and relevant industrial processes. We can highlight nanomaterials like polymers, composites or carbon-derived (graphite, graphene); ceramic materials, materials for additive manufacturing and 3D printing, waste and industrial subproducts, and materials for energy (batteries, fuel cells and gas storage)



ICAMCyL, A REFERENCE COMPETENCE CENTER



ADVANCED MINING TECHNOLOGIES

- Exploration
- Classification
- Processing



CIRCULAR ECONOMY

- Waste
- Recycling
- Valorisation



SUSTAINABILITY

- New production methods
- Eco-innovation
- Resource efficiency



NANOMATERIALS

- Carbon-based
- MOFs
- Composites
- Alloys



FABRICATION

- Additive manufacturing
- Pilot lines
- Industry

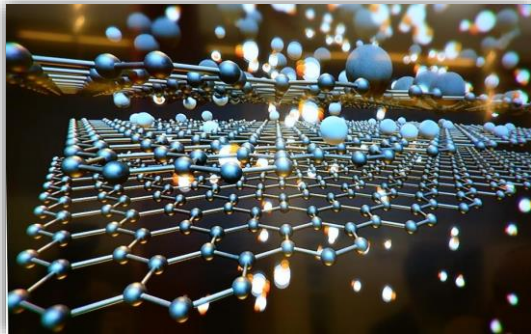


ENERGY & CLIMATE CHANGE

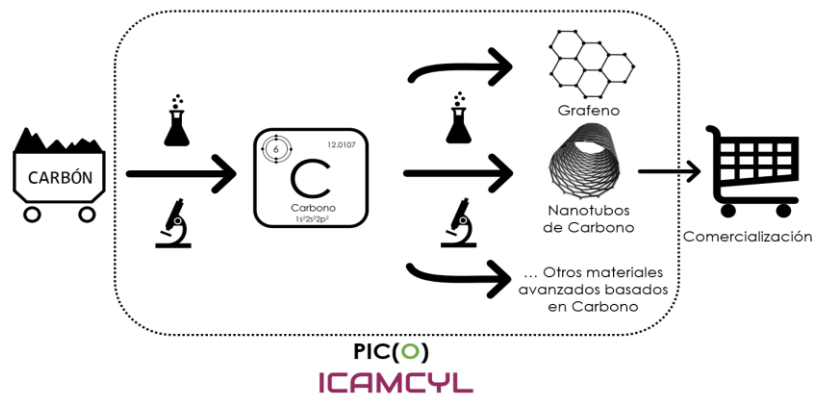
- Batteries /fuel-cells
- CO₂ capture
- Smart cities

POLES OF INNOVATION

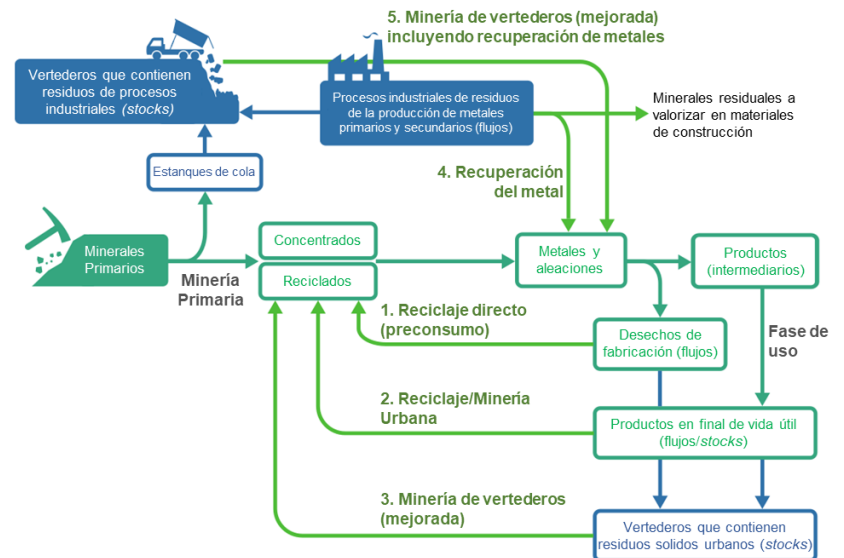
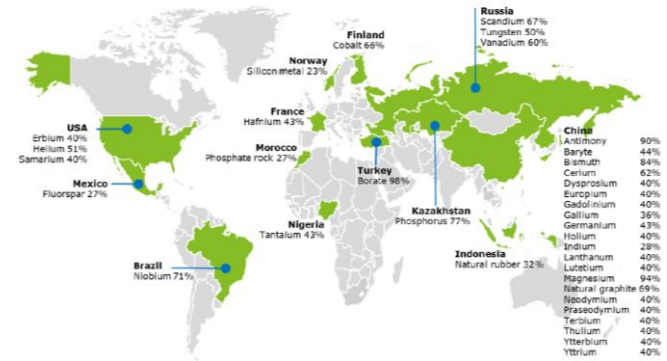
POLO PARA LA INNOVACIÓN DEL CARBÓN(O): RECUPERACIÓN DE LAS CUENCAS MINERAS MEDIANTE LA PRODUCCIÓN DE MATERIALES TECNOLÓGICOS DE BASE CARBONO A PARTIR DEL CARBÓN



Revitalizing the CyL region: recovering coal mining places by the creation of an innovation pole for carbon



POLO DE INNOVACIÓN Y RESTRUCTURACIÓN SOCIAL DE LAS CUENCAS MINERAS HACIA UNA NUEVA MINERÍA SOSTENIBLE



S3P – INDUSTRIAL MODERNIZATION FOR BATTERIES



CONSEJERÍA DE ECONOMÍA, INNOVACIÓN Y CIENCIA

Advanced materials for batteries



ELECTRO MOBILITY



STATIONARY ENERGY STORAGE

Aim of this S3P – Industrial Modernization is to **bridge the large gap** between research and industrial applications.

CURRENT ONGOING PROJECTS



Advanced materials solutions for next generation CSP tower systems



Mining and Metallurgy Regions of EU



Recovery of Tungsten, Niobium and Tantalum occurring as by-products in mining and processing waste streams



European Commission

Horizon 2020
European Union funding
for Research & Innovation

EXPLOREMAT:
Descubrimiento de materiales avanzados por métodos computacionales para la industria de Castilla y León



VALUECyL:
Valorización de las materias primas de los residuos de la actividad minera en Castilla y León



Solutions for Critical Raw Materials Under Extreme Conditions



Mining the European Anthroposphere



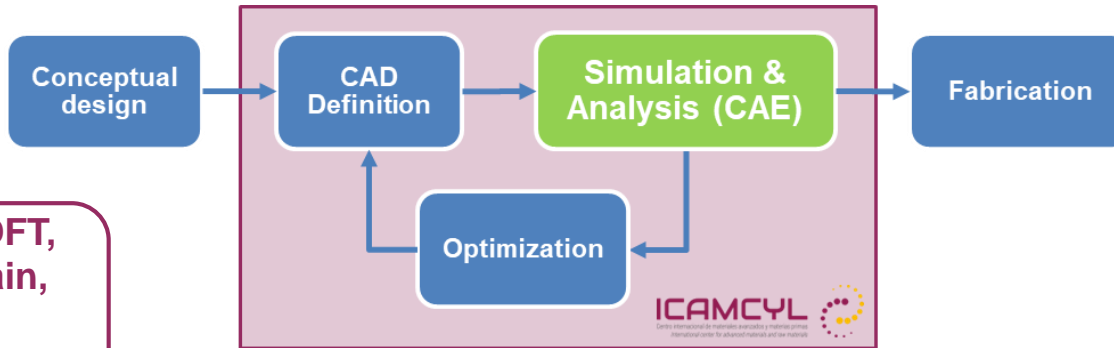
European Energy Poverty: Agenda Co-Creation and Knowledge Innovation



European Institute of Innovation & Technology

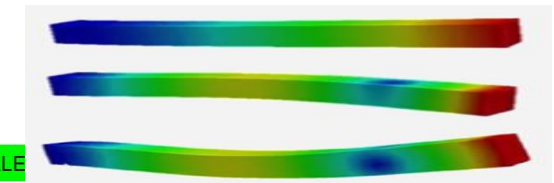
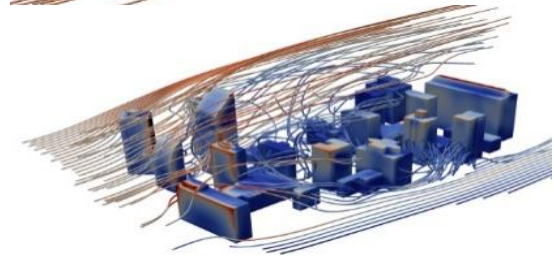
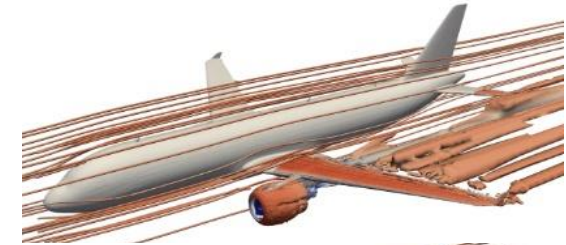
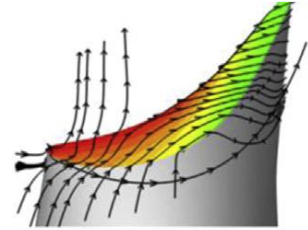
INNOCAT: Innovative CRM substitution technology

ENGINEERING & MATERIALS DESIGN DIVISION



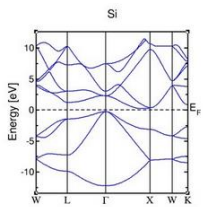
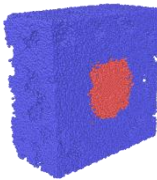
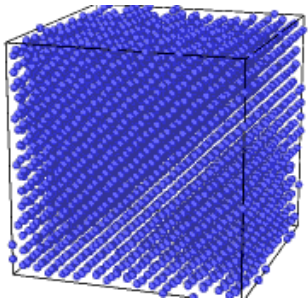
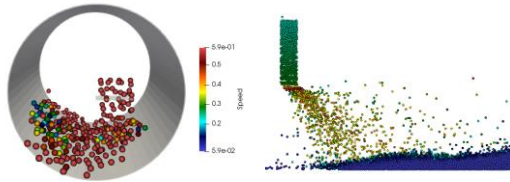
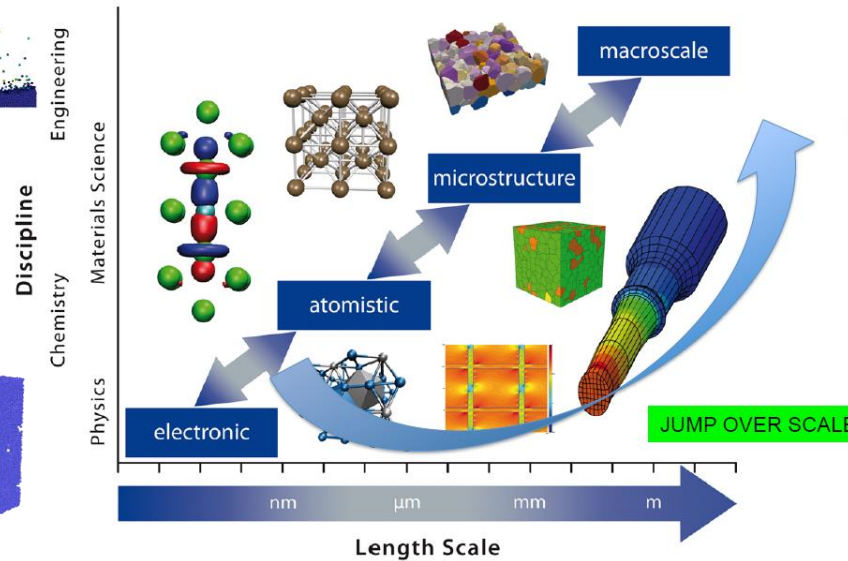
MULTISCALE (DFT, MD, Coarse Grain, DEM)

AbInit, LAMMPS, LIGGGHTS etc



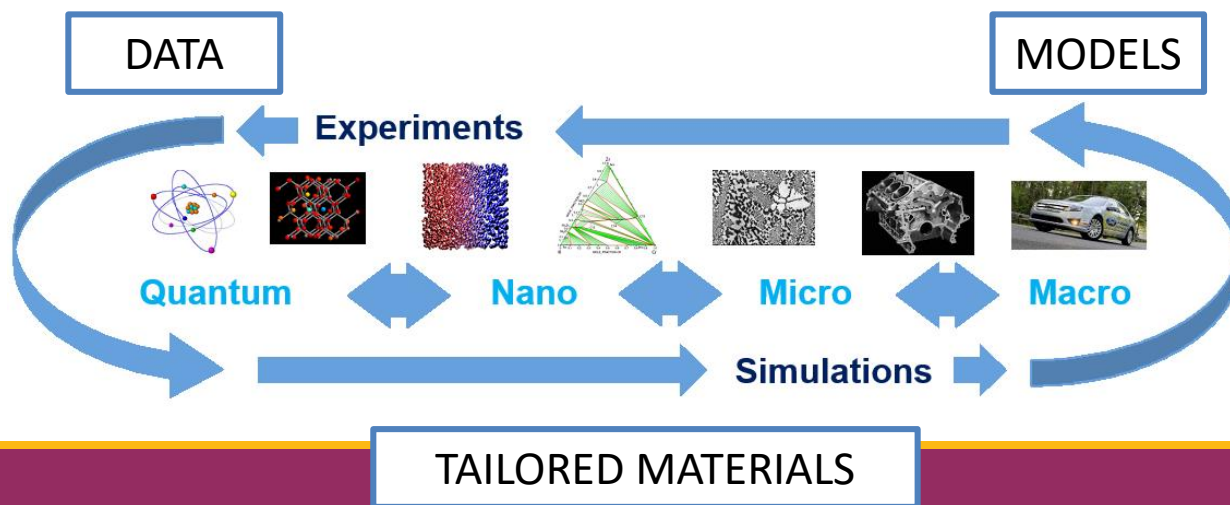
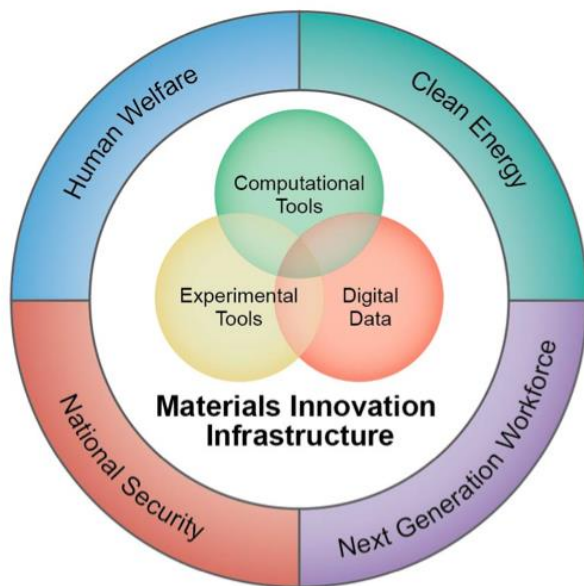
CFD / FEM
Ansys, openFOAM, Salome, etc.

MULTISCALE MODELLING APPROACH:



1. ABOUT ICAMCyL
- 2. ICAMCyL & MATERIALS DISCOVERY**
3. PREVIOUS PROJECTS
4. CURRENT PROJECTS

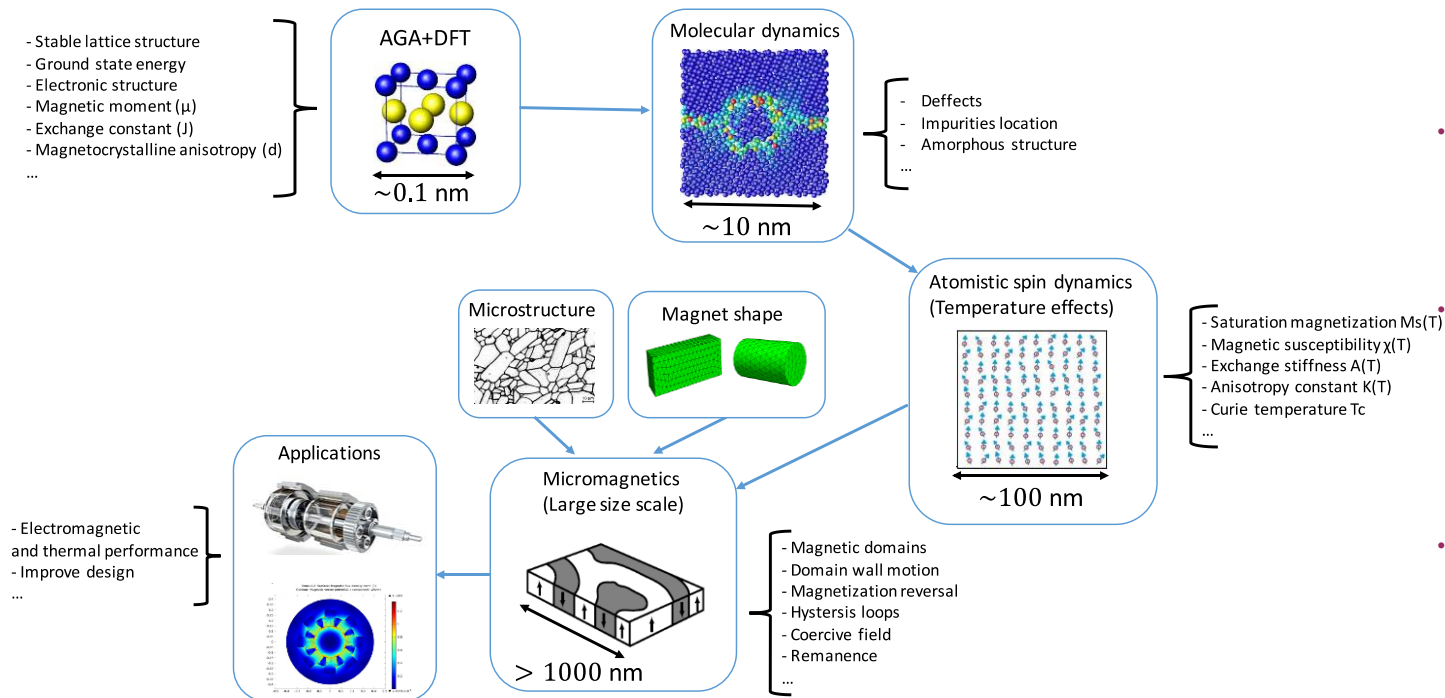
Materials Genome Initiative



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Multi-scale Modelling of Magnetic Materials

- Applying multiscale techniques to magnetic materials study with adaptative genetic algorithms
- High-throughput exploration of magnetic materials
- From nanostructure to final materials properties
- DFT -> MD -> Coarse Grain -> MacroScale

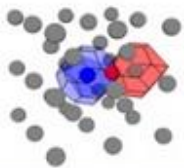


- Arapan S, Nieves P, Cuesta-López S. *A high-throughput exploration of magnetic materials by using structure predicting methods*. Journal of Applied Physics (2018) 123(8)
- Nieves P, Arapan S, Cuesta-López S; *An adaptive genetic algorithm approach for predicting magnetic structure suitable for high-performance permanent magnet development*; 2017 IEEE International Magnetics Conference, INTERMAG 2017 (2017)
- Nieves P, Arapan S, Cuesta-Lopez S; *Exploring the Crystal Structure Space of $\text{CoFe}_{1-x}\text{P}$ by Using Adaptive Genetic Algorithm Methods*; IEEE Transactions on Magnetics (2017) 53(11)

Tool for radiation damage in materials

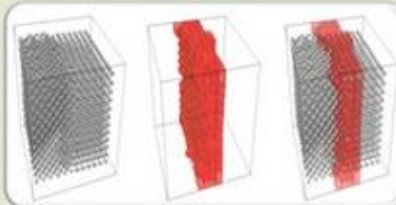
- Software tool base on the open source tool for Computational Fluid Dynamics (CFD) tool OpenFOAM
- Analysis of crystallographic structures in materials under radiation damage

RADAMATFOAM® analysis features



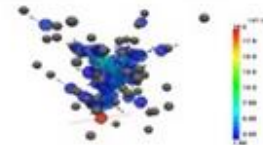
Bulk structures – WS Method

- Interstitials / Vacancies
- Clusterization of interstitials / vacancies
- Dumbbells



Nanostructures – Volume Method

- Cascades
- Grain Boundaries
- Interfaces



Visualization – VTK

- Scalar volume fields
- Deviation fields
- Defect diffusivity / rates
- Time / Space statistics

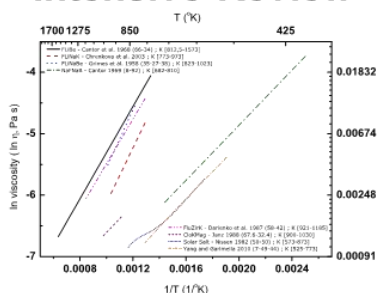
MATFOAM®. Ekhi Arroyo, Jordi Fradera, Santiago Cuesta, UBU. BU-90-12 11/07/2012.

RADAMAT-FOAM®
Radiation Damage In Materials For Openfoam.
E. Arroyo, J.Fradera, S. Cuesta, U. Burgos. BU 59-13. 03/05/2013.

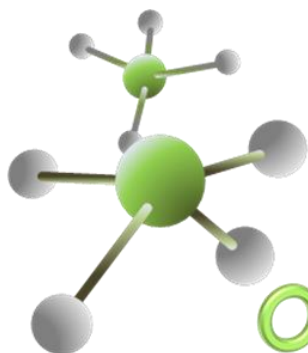
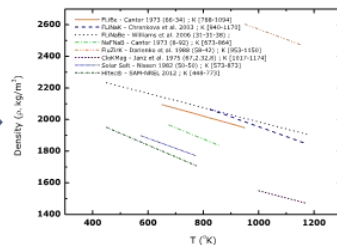
SOME EXAMPLES

Database for molten salt properties

Intensive Review



SCATERING, LACK OF DATA



On-line
Molten Salts
database Initiative

OMOSA[®]



Molten salts database for energy applications

R. Serrano-López, J. Fradera, S. Cuesta-López
Universidad de Burgos, Science and Technology Park, I+D+i Building, Room 63, Plaza Misael Bafuelos s/n, 09001 Burgos, Spain



Friendly web interface

14 Molten Salts
5 Properties



Quick access to data and references by filtering

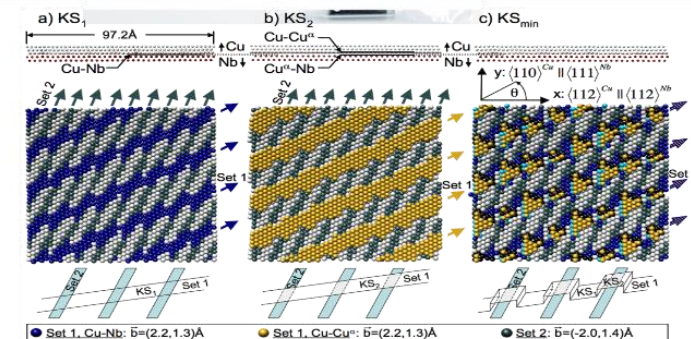
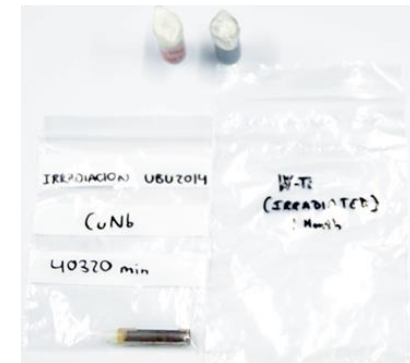
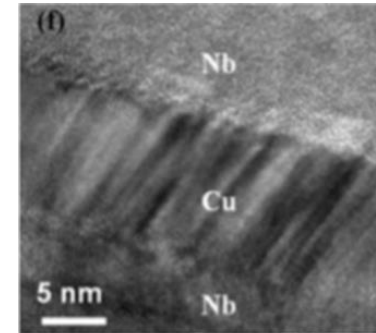
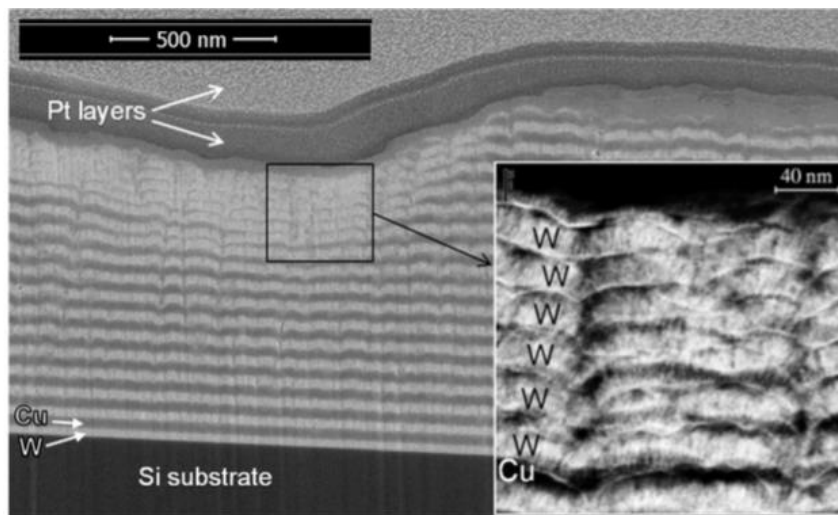


+ Properties prediction section
+ Uncertainties section

SOME EXAMPLES

From nano to multiscale damage in materials

- Nanostructured metallic multilayers
- Radiation damage healing and helium trapping properties
- Objective: changing the life-time perspective of radiation shielding
- CuNb & CuW look stable and arise as solid candidates to form multilayers, nano-structured metallic interfaces



IN COOPERATION with University of Cagliari (F. DELOGU et al)

From nano to multiscale damage in materials

CuNb Interface structure / dynamic information
Anticipation of KS1' structure

Massive simulations of radiation damage
cascades on Cu, Nb, W interfaces -> statistics

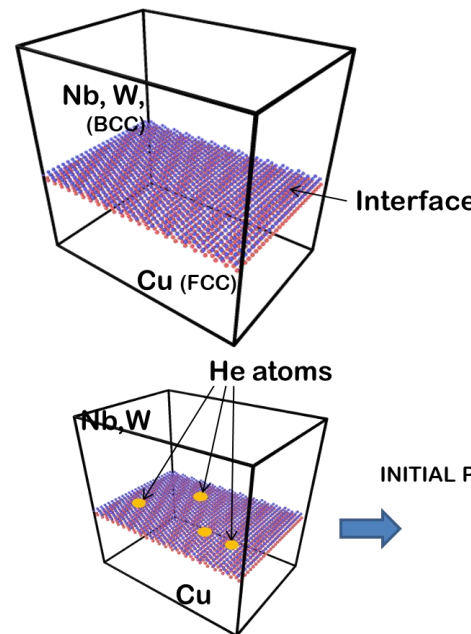


STATISTICS OF DEFECTS, DIFFUSION
COEFFICIENTS
IN FUNCTION OF ENERGY, PARTICLE, TIME



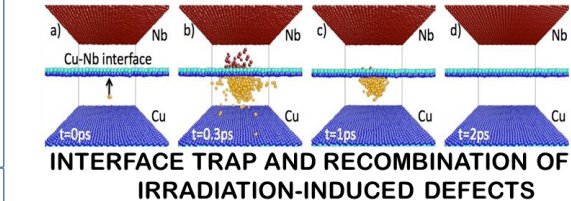
MODEL OF PREDICTION OF DAMAGE AT THE
MACROSCALE
DPA-ENGINEERING DESIGN MACROSCALE

- Ortún-Palacios J, Locci A, Fadda S, Delogu F, Cuesta-López S; *Role of Interface in Multilayered Composites under Irradiation: A Mathematical Investigation*; Advances in Materials Science and Engineering (2017) 2017
- Ortún-Palacios J, Locci A, Delogu F, Cuesta-López S; *Self-healing ability assessment of irradiated multilayered composites: A continuum approach*; Journal of Nuclear Materials (2018) 512 391-406

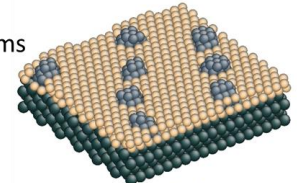


He DYNAMICS AT THE INTERFACE

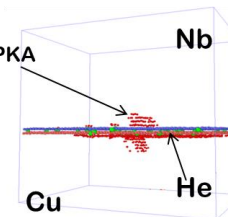
Self healing properties under radiation:



Ability to trap He atoms



INITIAL PKA



DISPLACEMENT CASCADE

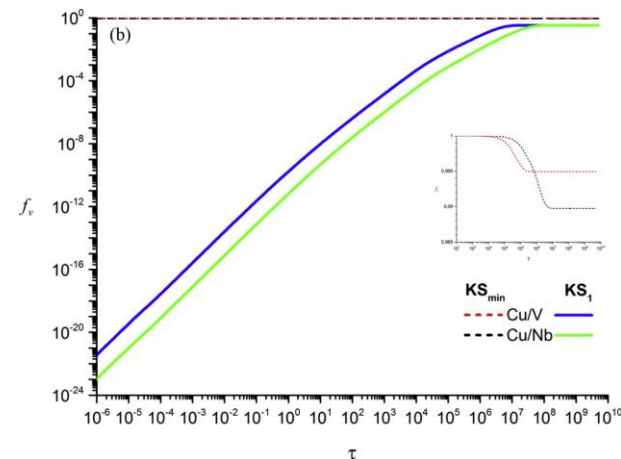
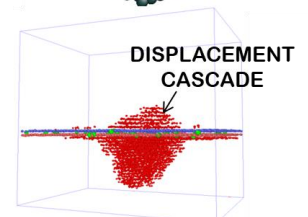


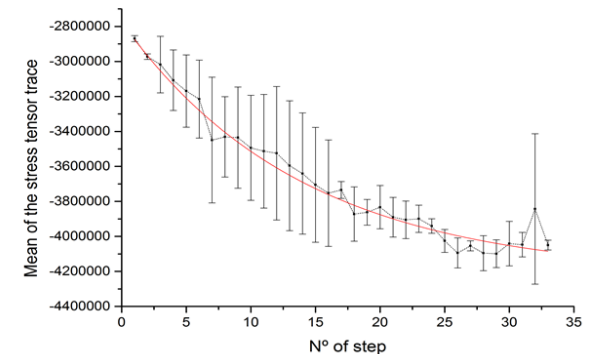
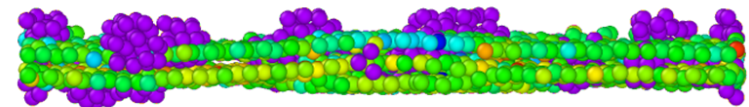
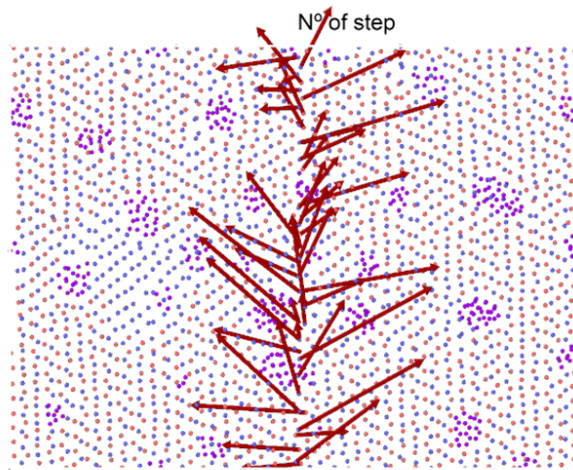
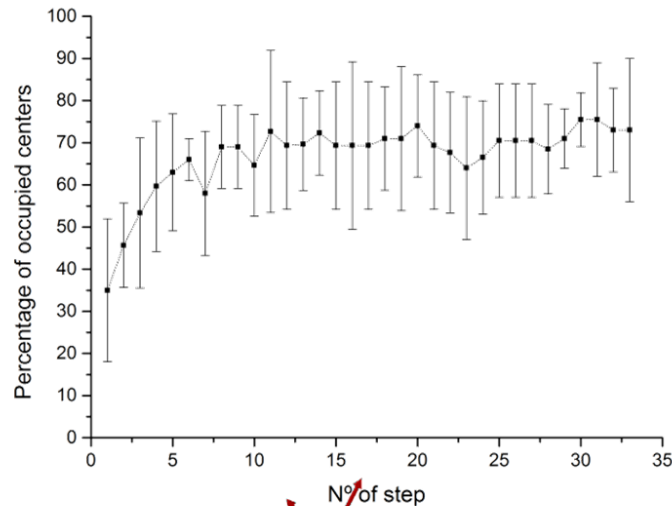
Fig. 9. Temporal profiles of (a) SIA and (b) vacancy trap occupation probability at the interface between metals x and y ($z = 1$).

From nano to multiscale damage in materials

He DYNAMICS AT THE INTERFACE

OUR SUCCESS-> WE PROVIDED:

- Confirmation of the existence of a engineering design structural factor relying on the molecular architecture
- A limit in the absorption based on the loose of mechanical properties
- The possibility to predict He confinement/adsorption rates over time



Atomistic view of nano-crashes, nanoindentation and impact phenomena

- Refractory materials study: niobium response against the impact of high energy debris
- MD study of microstructure after impact
- Damage response in niobium as shielding material, showing low energy dependence on dislocation density for debris energies over 10 keV
- Methodology can be extended to other refractory metals of industrial interest (tungsten, tantalum)



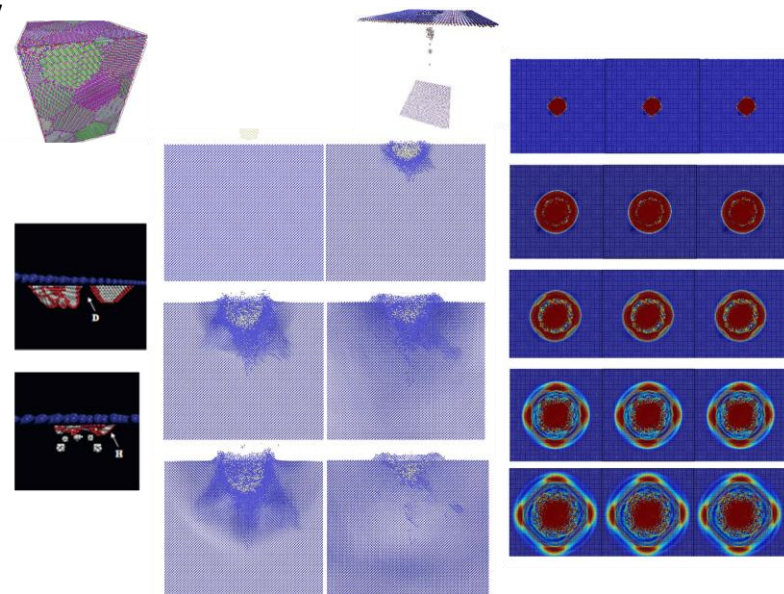
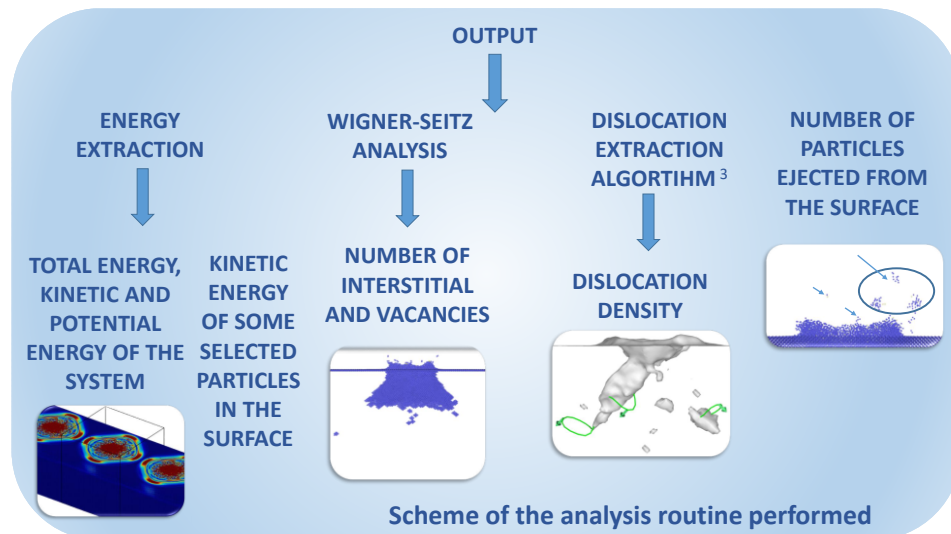
Nuclear Instruments and Methods in Physics
Research Section B: Beam Interactions with
Materials and Atoms

Volume 432, 1 October 2018, Pages 24-28



Theoretical study of the performance of
refractory materials for extreme conditions
applications

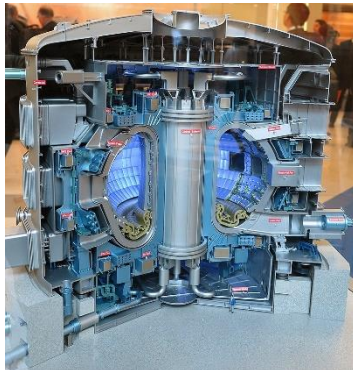
Claudia Pecoraro ^{a, b, c}, Santiago Cuesta-López ^{a, b, c, d, e}



SOME EXAMPLES

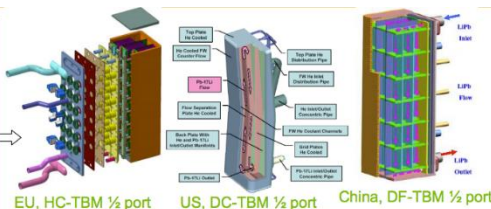
Liquid Metals in Nuclear Industry

- High relevance of liquid metals as coolants in nuclear industry: ITER project
- Behaviour of liquid metals can be studied with the help of multiscale modelling technique
- Previous experience in this field: developing interatomic potentials for Li-Pb, irradiation effects over coolants.
- Next step: He-Li-Pb interatomic potential



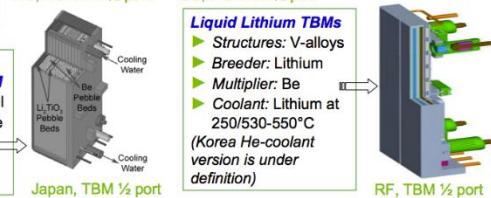
Lithium-Lead (LL) TBMs

- Structures: F/M Steel
- Multiplier/Breeder: Eutectic Pb-16Li
- Coolant: He at 8 MPa, 300/500°C alone (HCLL) or with LiPb at 460/700°C (DCLL, DFLL)



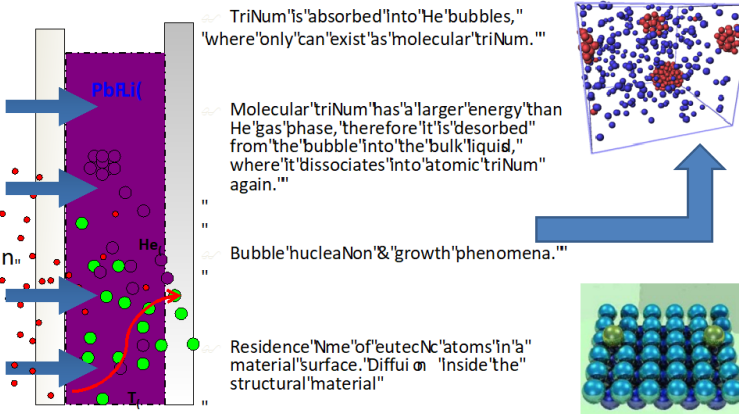
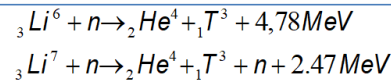
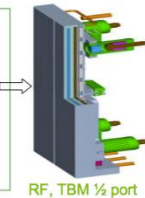
Water-Cooled Ceramic Breeder TBM

- Structures: FM Steel
- Multiplier: Be pebble
- Breeder: Li_2TiO_3
- Coolant: He at 8 MPa, 300/500°C



Liquid Lithium TBMs

- Structures: V-alloys
- Breeder: Lithium
- Multiplier: Be
- Coolant: Lithium at 250/530-550°C (Korea He-coolant version is under definition)



- Fraile, A., Cuesta-López, S., Caro, A., Schwen, D., Manuel Perlado, J. Interatomic potential for the compound-forming Li-Pb liquid alloy (2014) *Journal of Nuclear Materials*, 448 (1-3), pp. 103-108.
- Fradera, J., Cuesta-López, S. Impact of nuclear irradiation on helium bubble nucleation at interfaces in liquid metals coupled to permeation through stainless steels (2014) *Fusion Engineering and Design*, 89 (1), pp. 16-24.
- Fradera, J., Cuesta-López, S. The effect of a micro bubble dispersed gas phase on hydrogen isotope transport in liquid metals under nuclear irradiation (2013) *Fusion Engineering and Design*, 88 (12), pp. 3205-3214.
- Fradera, J., Cuesta-López, S. Nucleation, growth and transport modelling of helium bubbles under nuclear irradiation in lead-lithium with the self-consistent nucleation theory and surface tension corrections (2013) *Fusion Engineering and Design*, 88 (12), pp. 3215-3223.
- Fraile, A., Cuesta-López, S., Iglesias, R., Caro, A., Perlado, J.M. Atomistic molecular point of view for liquid lead and lithium in Nuclear Fusion technology (2013) *Journal of Nuclear Materials*, 440 (1-3), pp. 98-103.

SOME EXAMPLES

Using CFD technology to make the bridge in design:
permeation of helium through fusion steels at interfaces in
liquid metals

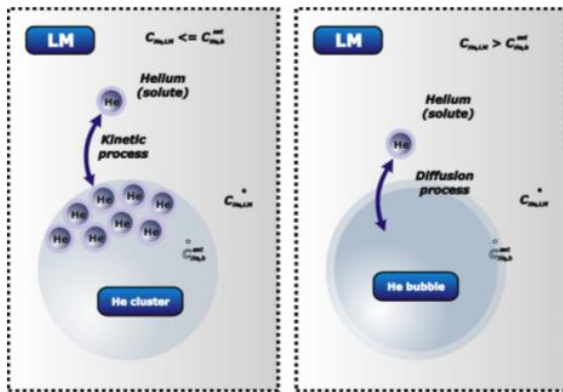


Fig. 1. Kinetic growth model (left) and diffusion growth model (right) showing when a model is used depending on the He concentrations.

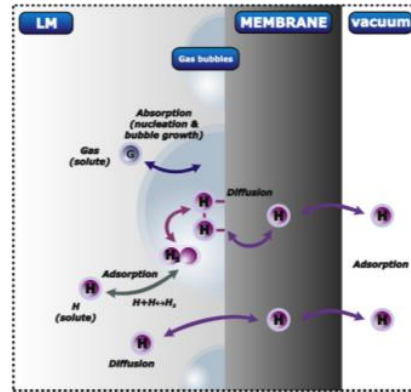


Fig. 1. Hydrogen isotope and helium transport phenomena in a permeation system.

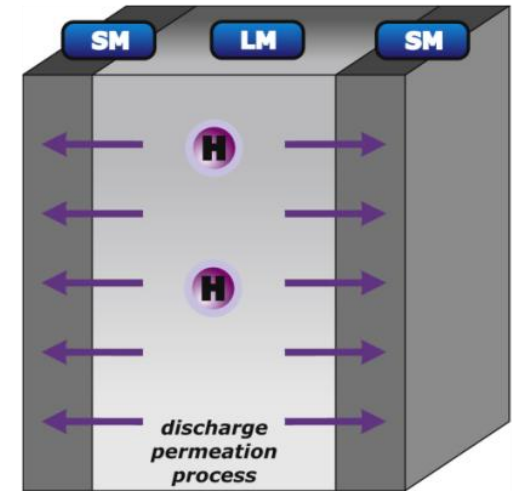


Fig. 3. One dimensional case configuration. Central LLE slab T discharge process through permeation process.

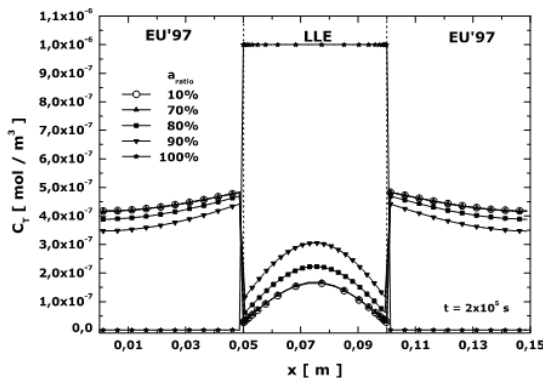


Fig. 8. Concentration profiles at different a_{ratio} for $t = 2 \times 10^5$ s.

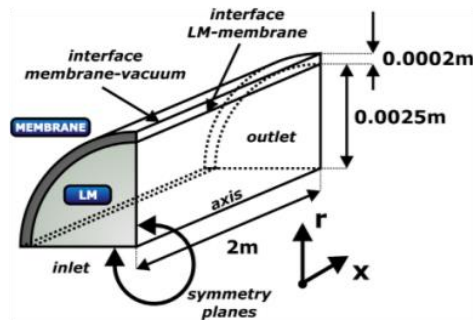
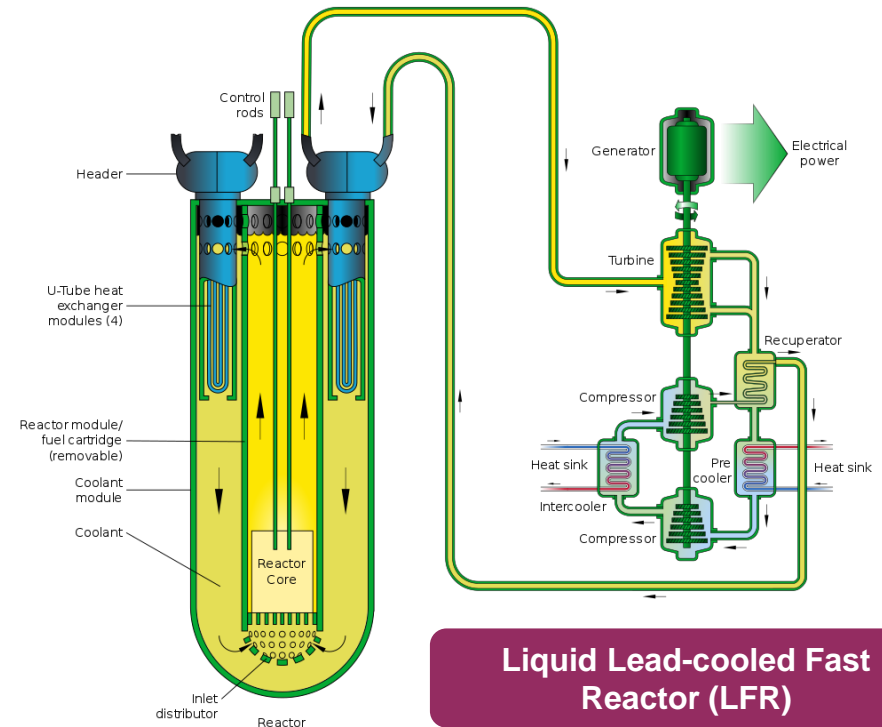
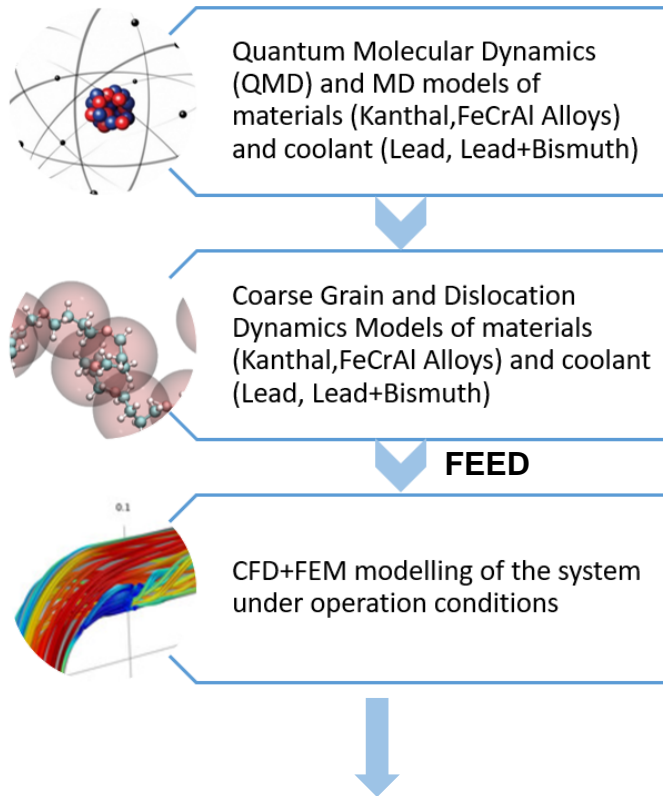


Fig. 9. 3D pipe configuration for the pipe CFD simulation. Symmetry planes are used to save computational resources.

- Fradera, J., Cuesta-López, S. Impact of nuclear irradiation on helium bubble nucleation at interfaces in liquid metals coupled to permeation through stainless steels (2014) *Fusion Engineering and Design*, 89 (1), pp. 16-24.
- Fradera, J., Cuesta-López, S. The effect of a micro bubble dispersed gas phase on hydrogen isotope transport in liquid metals under nuclear irradiation (2013) *Fusion Engineering and Design*, 88 (12), pp. 3205-3214.

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- 4. CURRENT PROJECTS**

Multiscale modelling of ODS alloys – FeCrAl-Pb interaction



OUR APPROACH

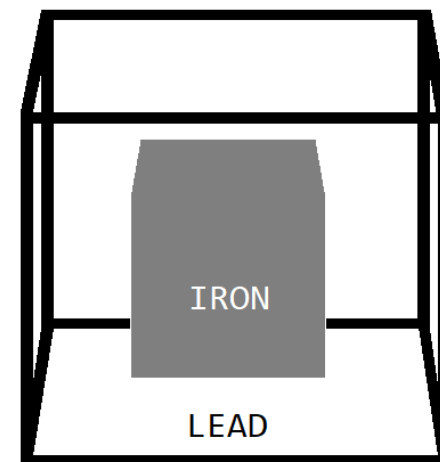
1. MD model for liquid metal corrosion
2. Coupled CFD+FEM model for liquid metal refrigeration

Objective:

- FeCrAl-Pb corrosion study
- Pure diffusion approach to corrosion

Steps:

1. Build model with Fe and Pb
2. Study diffusion coefficients dependence with T
3. Build FeCrAl and study diffusion of components
4. Build Alumina and study diffusion



LJ potentials

$$V(r) = 4\epsilon \left[\left(\frac{\sigma}{r} \right)^{12} - \left(\frac{\sigma}{r} \right)^6 \right]$$



More elements can
be considered



Less accurate

MEAM potentials

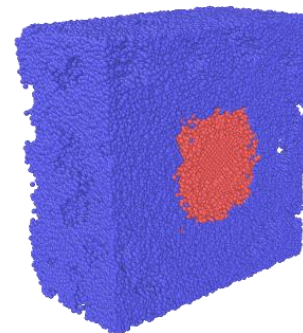
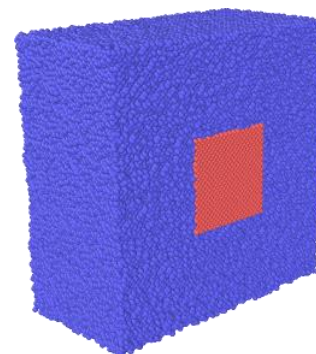
$$E_{tot} = \sum_i \left[F_i(\bar{\rho}_i) + \frac{1}{2} \sum_{j(\neq i)} S_{ij} \phi_{ij}(R_{ij}) \right]$$



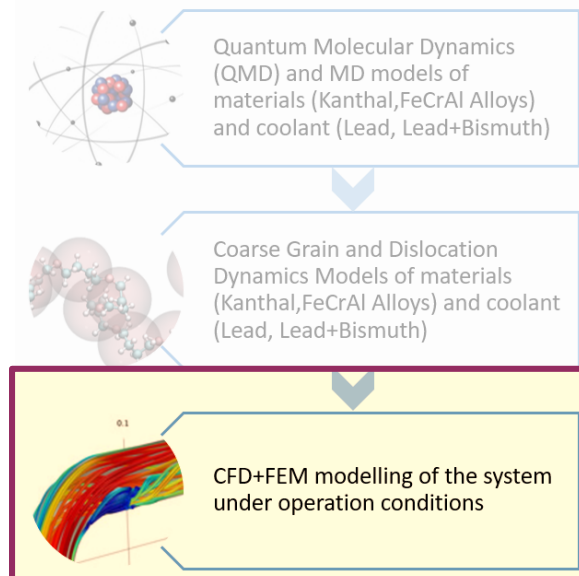
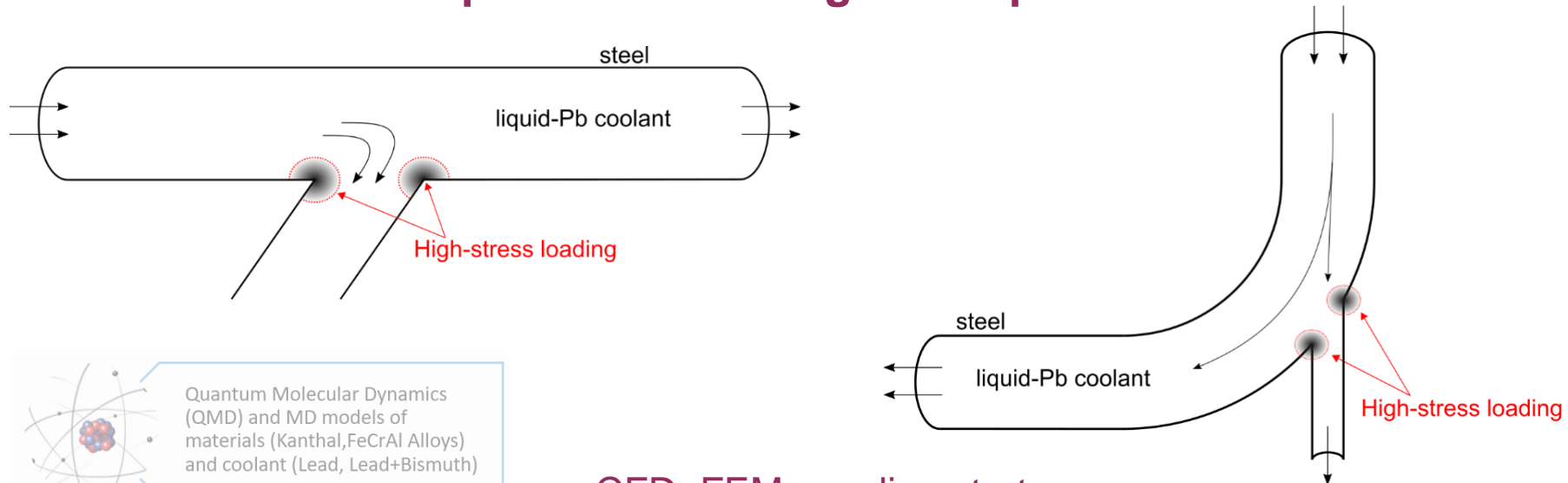
More accurate



Only available for a
few elements



The physical problem now at a macros-scale level: **appearance of weak points under long-term operation**



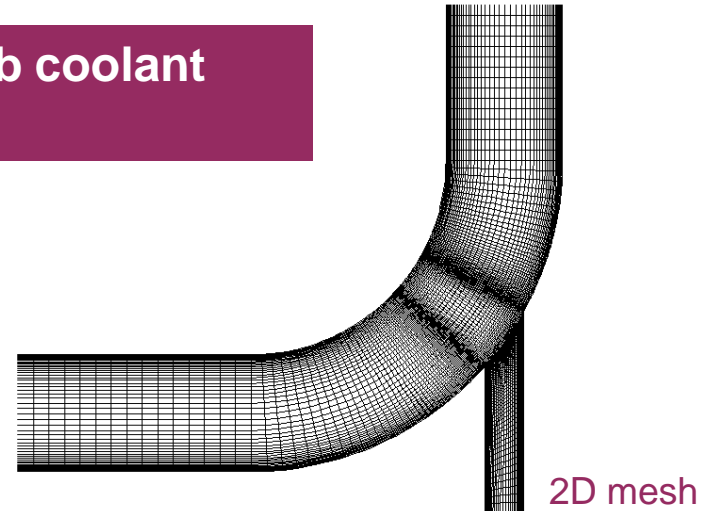
CFD+FEM coupling strategy:

- Interactions between liquid-Pb coolant and Kanthal pipe
- High shear-stress and thermal loading near joints
- How can these additional loads affect the aging and thus the operability of the solid material?
- Steady state vs. Transient operation

CFD simulations to investigate the liquid-Pb coolant flow problem

1. Mesh Generation

- Structured grids
- Low y^+ to resolve with enough accuracy hydrodynamic and thermal boundary layer
- Grid sensitivity study

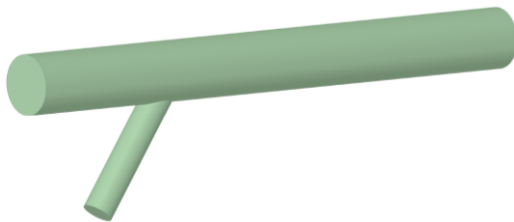


2. Numerical Simulations

- ANSYS Fluent solver (2D + 3D with thickness at later stage)
- Steady state + time-dependent simulations with heat transfer and buoyancy-driven effects
- Realistic internal cooling flow conditions: Reynolds number range $Re_D \in (10^4 - 10^6)$, fully turbulent flow
- Working fluid: pure liquid-Pb (with the possibility to include impurities at a later stage from molecular analysis)

3. Post-processing

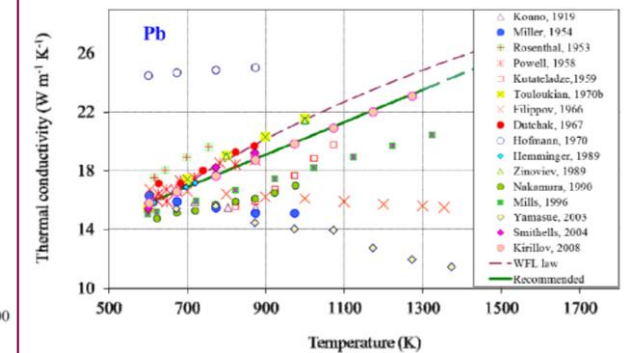
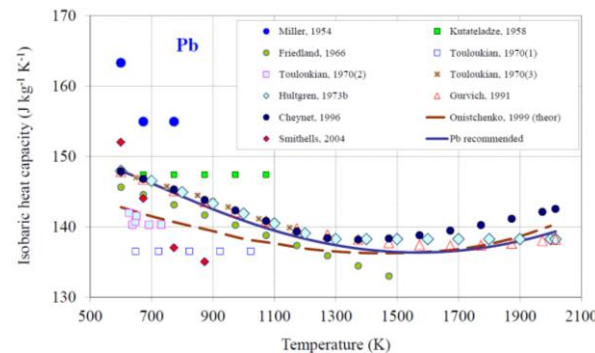
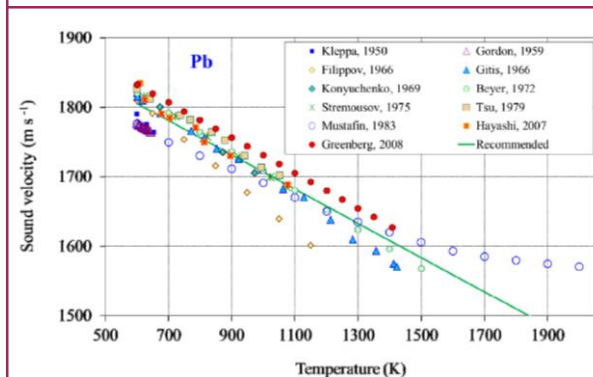
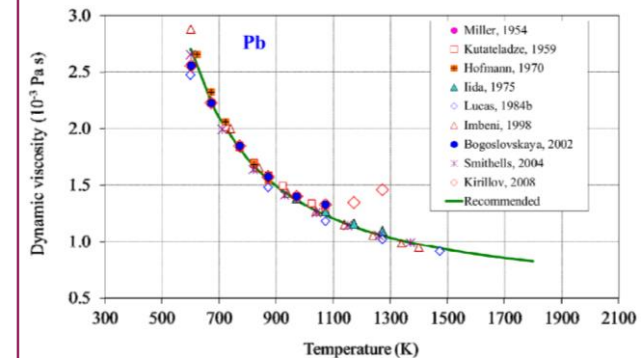
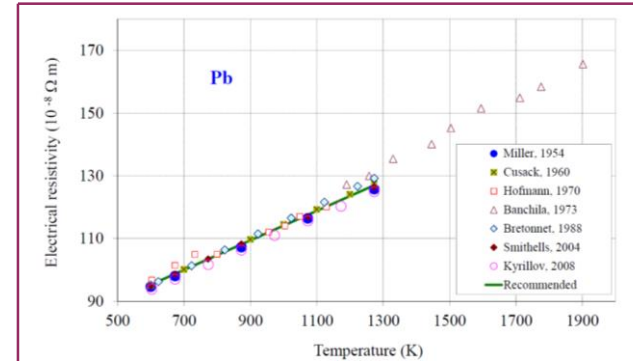
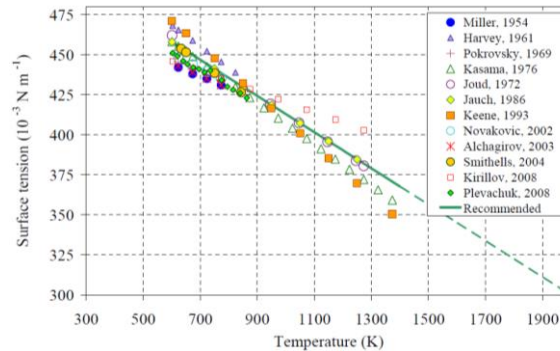
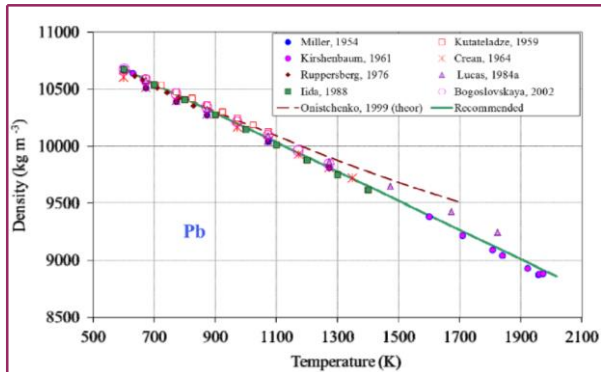
- Wall data regarding shear stress, friction coefficient, heat flux, temperature, among others



MACROSCALE MODEL: FeCrAl-Pb interaction

Correlations extracted from:

Vitaly Sobolev, *Database of thermophysical properties of liquid metal coolants for GEN-IV – Sodium, lead, lead-bismuth eutectic (and bismuth)*, Scientific Report SCK-CEN-BLG-1069, Belgian Nuclear Research Centre, 2011.



MACROSCALE MODEL: FeCrAl-Pb interaction

KANTHAL®

Part of Sandvik Group



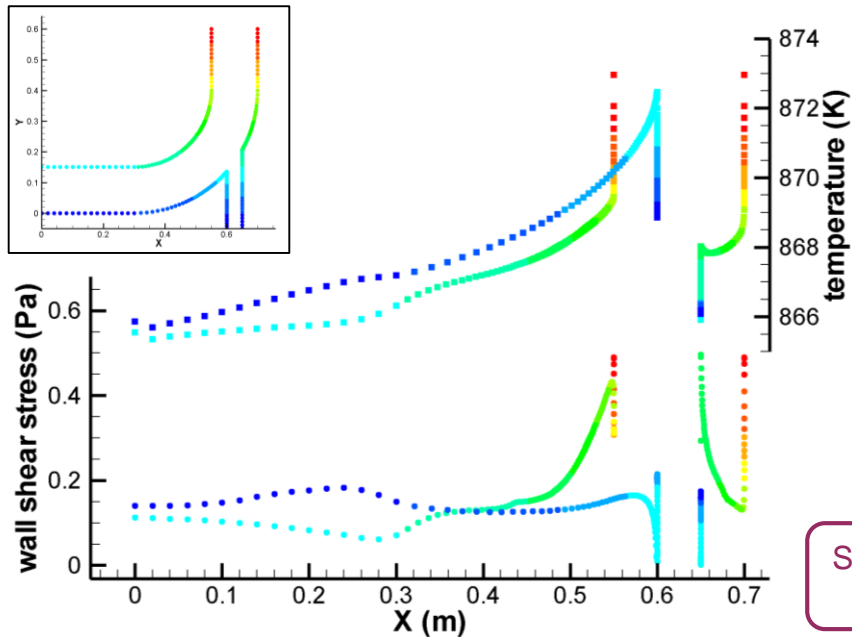
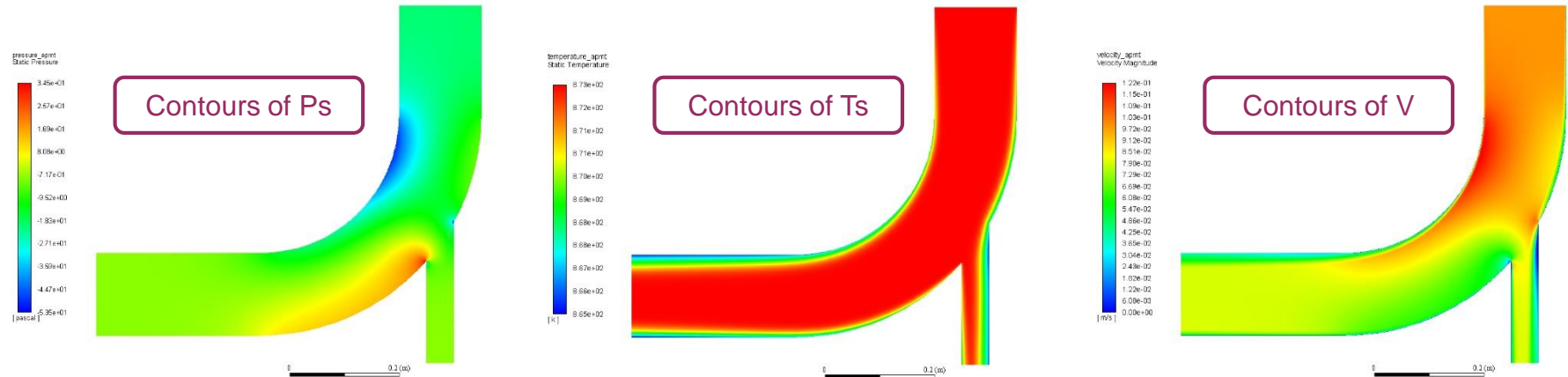
	C %	Si %	Mn %	Mo %	Cr %	Al %	Fe %
Nominal composition				3.0	21.0	5.0	Bal.
Min	-	-	-		20.5	-	
Max	0.08	0.7	0.4		23.5	-	

Temperature °C	Thermal Expansion Coeff. x 10 ⁻⁶ / K
20 – 250	12.4
20 – 500	13.1
20 – 750	13.6
20 – 1000	14.7
20 – 1200	15.4

Density g/cm³							7.25							
Electrical resistivity @ 20 °C Ω mm²/m							1.40							
Poisson's ratio							0.30							
T (°C)	20	100	200	300	400	500	600	700	800	900	1000	1100	1200	1300
Young's modulus (GPa)	220	210	205	-	190	-	170	-	150	-	130	-	-	-
Temperature factor of resistivity (Ct)	-	1.00	1.00	1.01	1.01	1.01	1.02	1.02	1.02	1.03	1.03	1.03	1.03	1.04
Thermal conductivity (x 10⁻⁶/K)	11	-	-	-	-	-	21	-	23	-	27	-	29	-
Specific heat capacity (kJ kg⁻¹ K⁻¹)	0.48	-	0.56	-	0.64	-	0.71	-	0.67	-	0.69	-	0.70	-
Melting point							1500 °C							
Magnetic properties							The material is magnetic up to approximately 600 °C (Curie point)							
Emissivity - fully oxidized material							0.70							

Data extracted from Kanthal datasheet and implemented in the solver

Preliminary CFD results – 2D steady & turbulent liquid-Pb flow



Parameters for CFD simulations	Values
Tube diameter (mm)	150
Inlet temperature (K)	873
Reynolds number (-)	10^5
Static Pressure (MPa)	0.1
Turbulence intensity (%)	5.0
Turbulent viscosity ratio (-)	5.0
Wall heat transfer coeff. (W/m ² .K)	20.0

Shear stress and wall temperature
(coloured by the y-direction)

Following step:
CFD-FEM coupling to investigate the liquid-Pb coolant flow problem

CFD solver



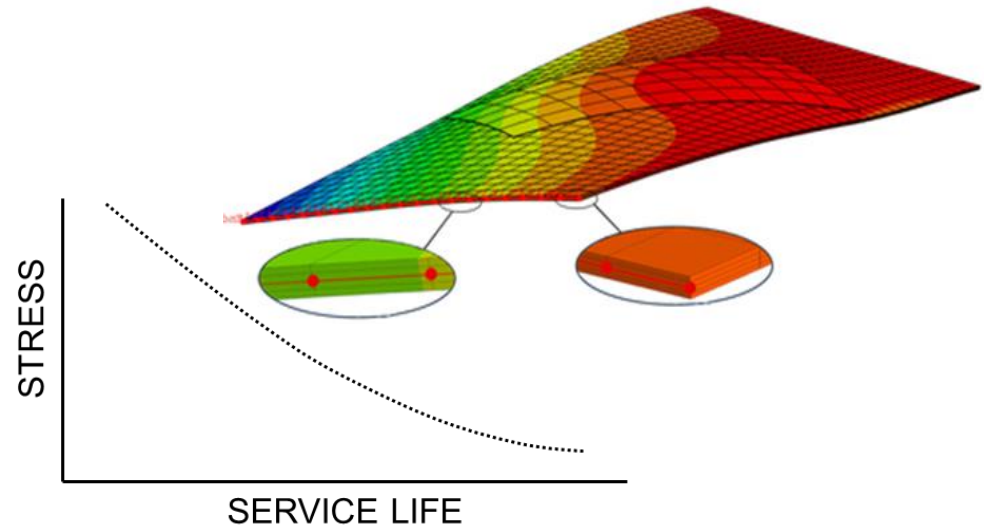
Coupled FEM

Hydrodynamic and thermal loading
on the solid material extracted from
CFD simulations

Structural study – thermal and
stress analysis

GOALS:

- Investigate the variation of the mechanical properties of the Kanthal for real stress loading conditions
- Estimate with higher precision the service life of the Kanthal for liquid-Pb cooling nuclear applications



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